

Engineered Zircaloy Cladding Modifications for Improved Accident Tolerance of LWR Fuel

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E N G I N E E R I N G A T I L L I N O I S



US DOE NEUP ATF-IRP—UIUC Prime



Objectives and Background

- Response to large amounts of H generation due to high temperature oxidation of Zr-base cladding
- A major concern both Post-Three Mile Island and Post-Fukushima
- Several US and International Programs

Issues: Extensive experience with Zr-based cladding

- Irradiation performance, processing experience, ...
- Qualification issues, ...
- Very difficult to move to a new cladding material

Approach: Examine Zr-based cladding with improved high temperature oxidation resistance



Participants

Project Director: Brent J. Heuser/University of Illinois

Partner Institutions: University of Illinois: Brent J. Heuser, Tomasz Kozlowski, Rizwan Uddin, James F. Stubbins, Dallas R. Trinkle, Robert S. Averback

University of Michigan: Thomas J. Downar, Gary S. Was

University of Florida: Yong Yang, Simon R. Phillpot

Idaho National Laboratory: Piyush Sabharwall, Michael V. Glazoff, Jason D. Hales

University of Manchester: Michael Preuss, Simon M. Pimblott, M.G. Burke, E. Jimenex-Melero, Fabio Scenini, Philip J. Withers

ATI Wah Chang: Melissa Martinez, Greg Vignoul



UIUC AFT-IRP Philosophical Approach

U.S. LWR assets are safe, well-maintained, and well-operated.

Zr-based cladding performs well in LWRs under normal operational conditions.

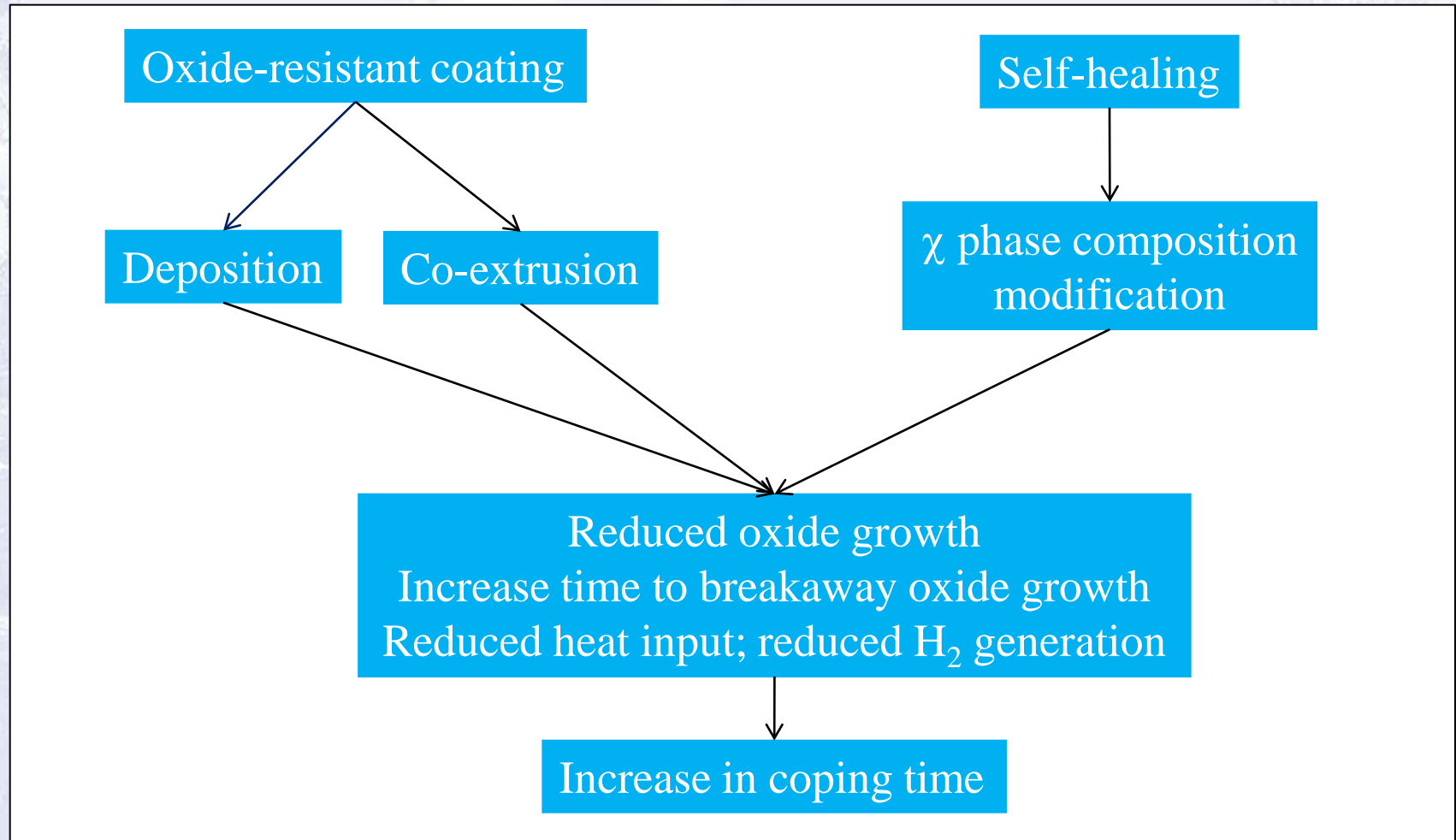
Extensive performance data base with respect to normal and transient conditions.

Regulatory approval/industry acceptance for modified Zr-based cladding
path of least resistance.

Modifications of Zr-based cladding can lead to ATF without significant
impact on performance under normal operational conditions.



Two Solution Pathways to Mitigate Accelerated Oxidation and H₂(g) Production



Self-healing pathway

ATI Wah Chang
(intermediate stock supplier)

Zircaloy

Composition^a (weight %) of various zirconium alloys.

Alloys	Tin	Iron	Chromium	Nickel	Niobium
Zircaloy-1	2.50	-	-	-	-
Zircaloy-2	1.50	0.12	0.10	0.05	-
Zircaloy-3A	0.25	0.25	-	-	-
Zircaloy-3B	0.50	0.40	-	-	-
Zircaloy-3C	0.50	0.20	-	0.20	-
Zircaloy-4	1.50	0.20	0.10	-	-
ZIRLO	1.02	0.10	-	-	1.01
M5 [®]	-	0.05	0.015	-	1.0
É110	-	-	-	-	0.95-1.05
É125	-	-	-	-	2.20-2.60
É635	1.1-1.3	0.3-0.4	-	-	0.95-1.05
OPT ZIRLO	0.66	0.11	-	-	1.04
X5A (AXIOM)	0.5	0.35	0.25	-	0.3

^a Remainder zirconium.

χ phases

χ phases with additives:
Si, Al, Mo, Cr

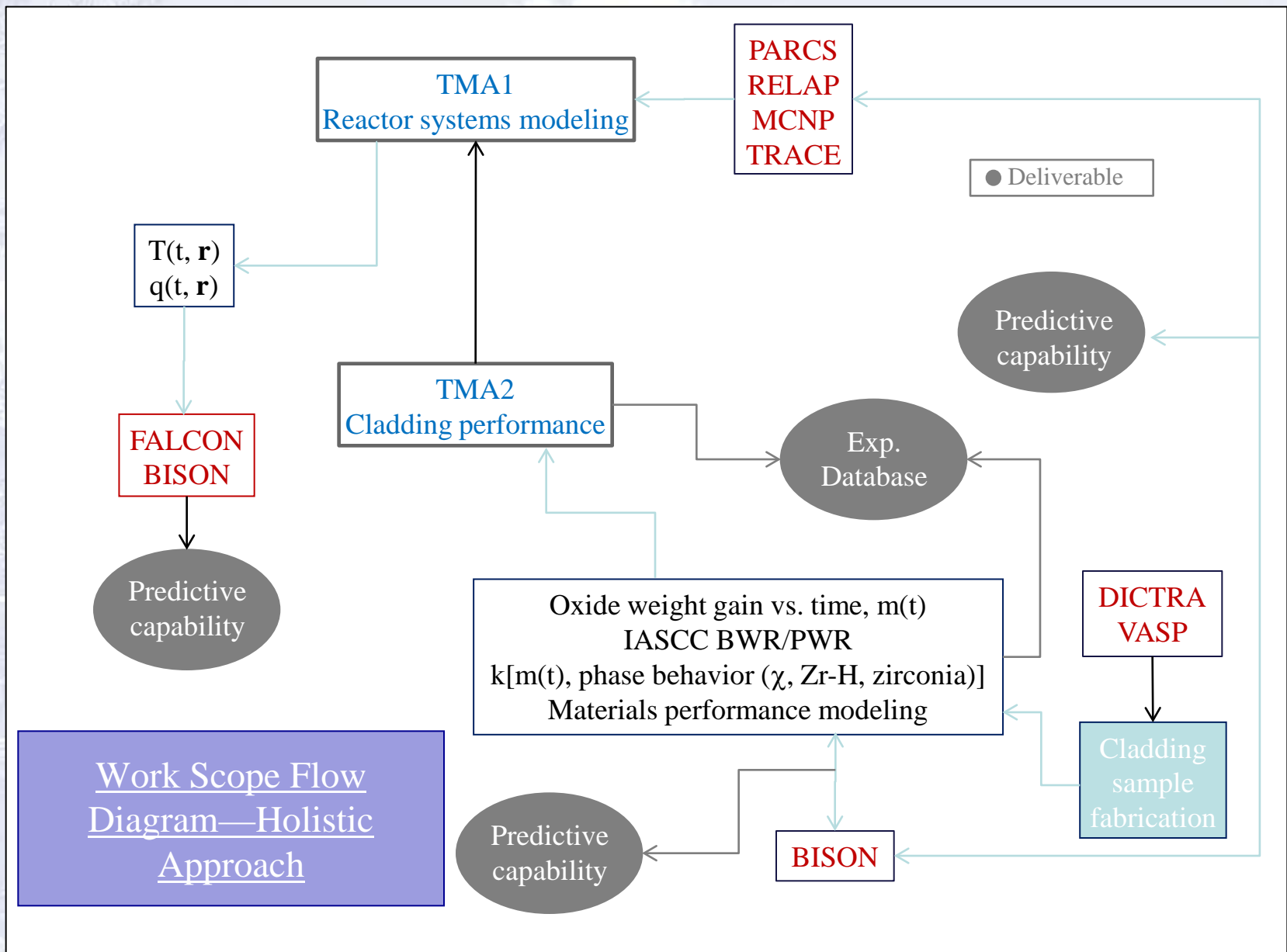
Additives segregate to free surface;
incorporated into oxide;
lower growth kinetics;
longer onset to accelerated
oxide growth.

χ precipitates dissolve
~900 C

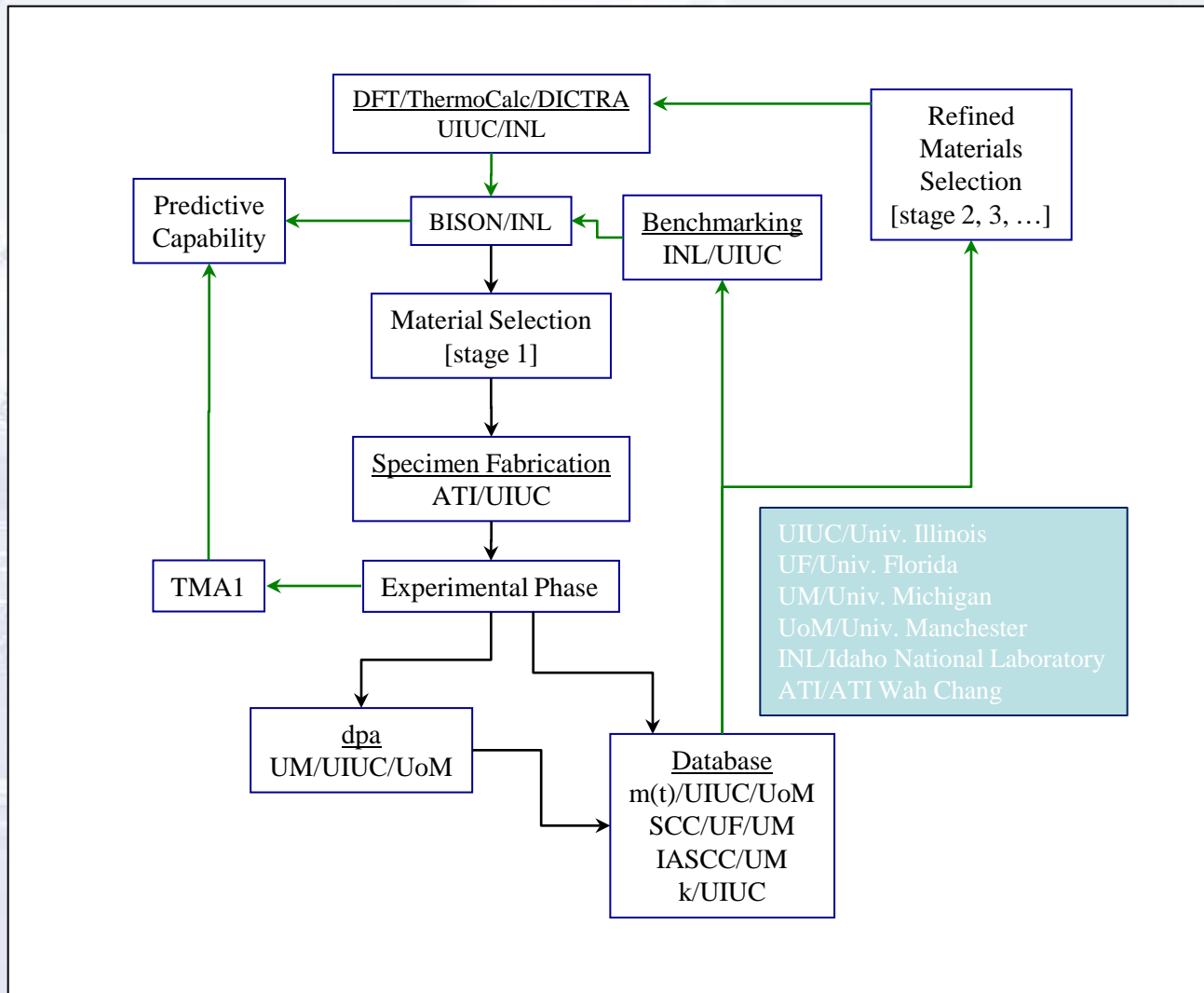
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Stable χ precipitates at
LWR operating T



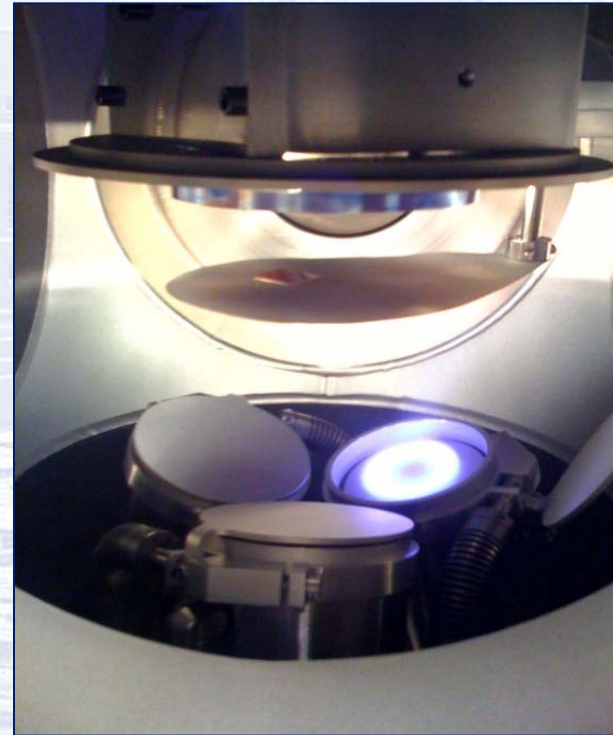


Participation Interconnections



Experimental Capabilities—Sample Fabrication

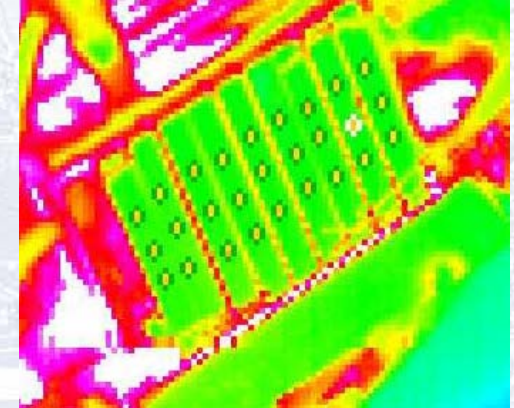
Sputter deposition: U. Illinois
Cladding fabrication: ATI Wah Chang



Experimental Capabilities—In-service corrosion

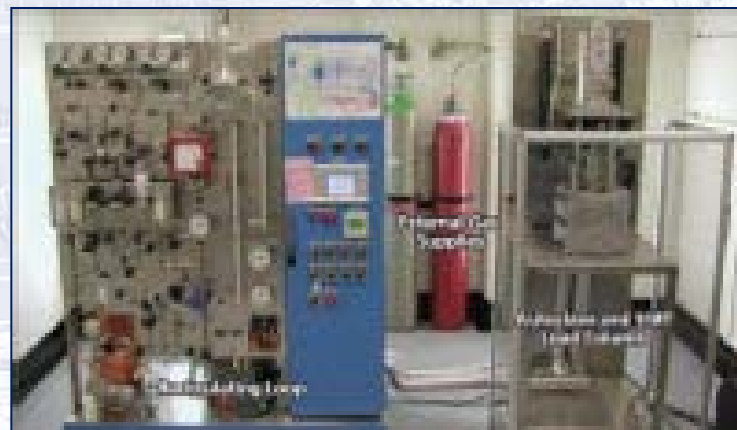
Autoclave capability: U. Michigan, U. Florida, U. Manchester

Ion accelerator capability: U. Illinois, U. Michigan, U. Manchester



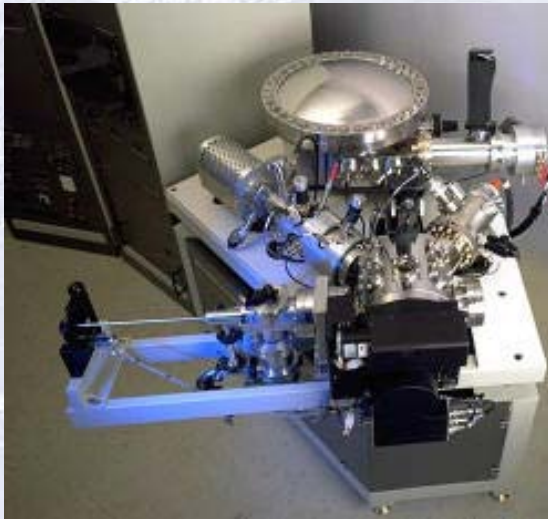
Experimental Capabilities—Off-normal oxidation

TGA: U. Illinois, U. Manchester



Experimental Capabilities—Microanalytical Characterization

Microanalytical: U. Illinois (FS-MRL), U. Michigan, U. Florida, U. Manchester
AES, TOF-SIMS, XPS, FIB, X-ray based techniques, TEM, SEM, AFM, ...
ANL: IVEM, APS



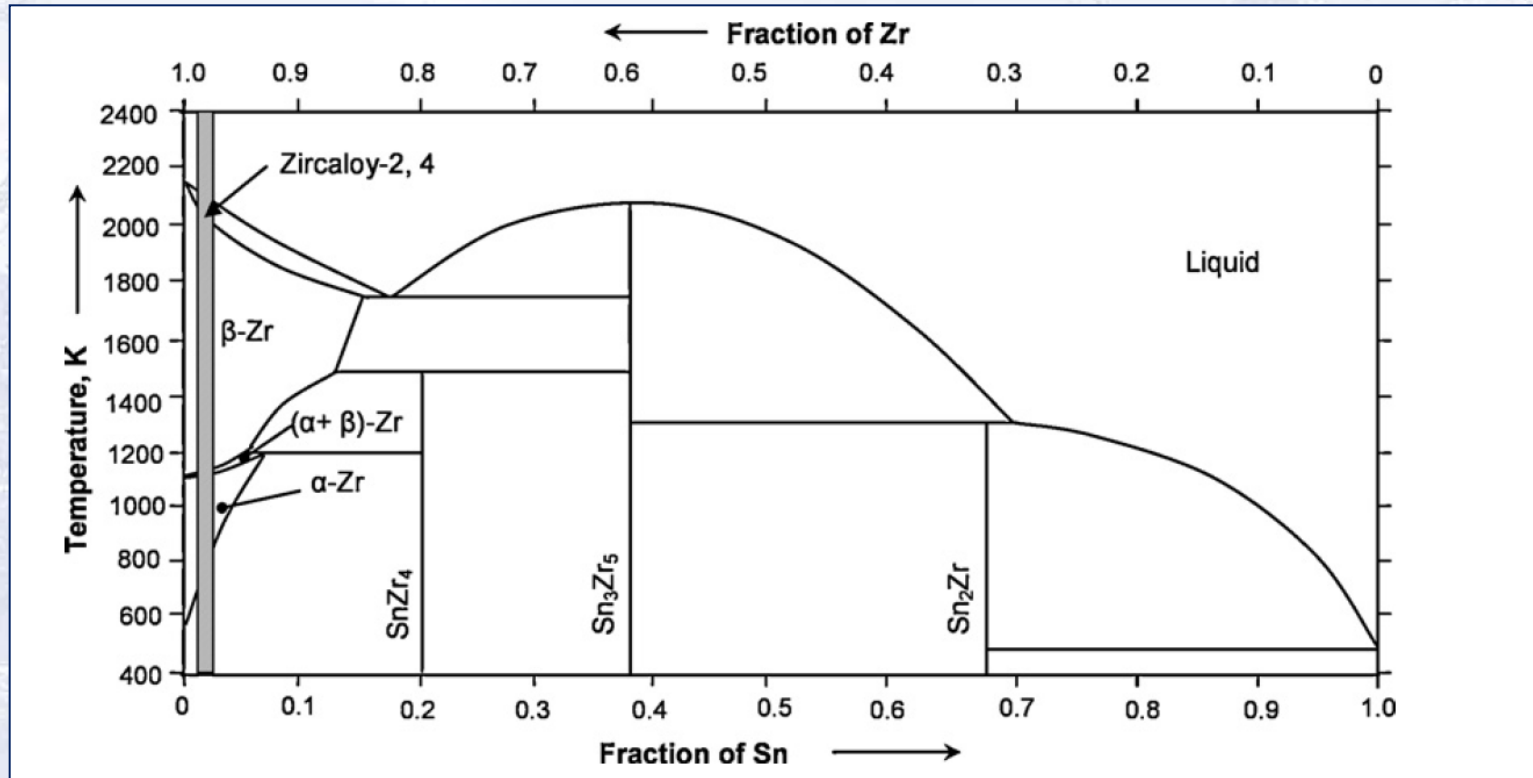
U.S. DOE NEUP/U.K. RCEP Investment

Table 5. Summary of budget allocation for IRP partners.

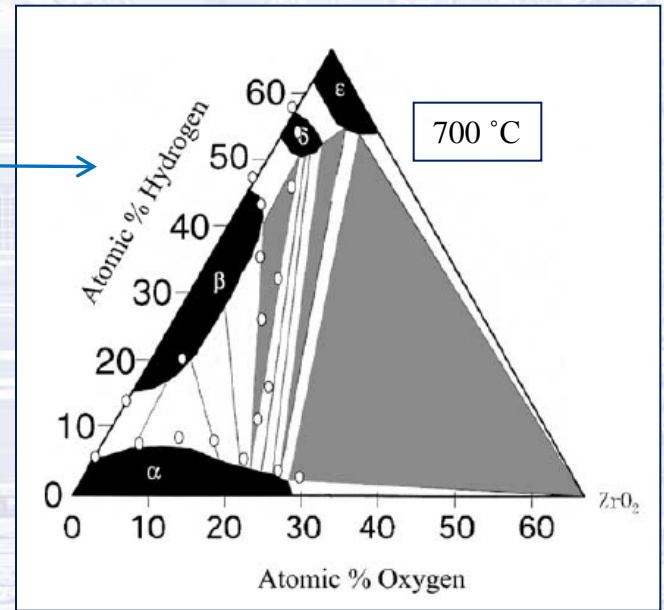
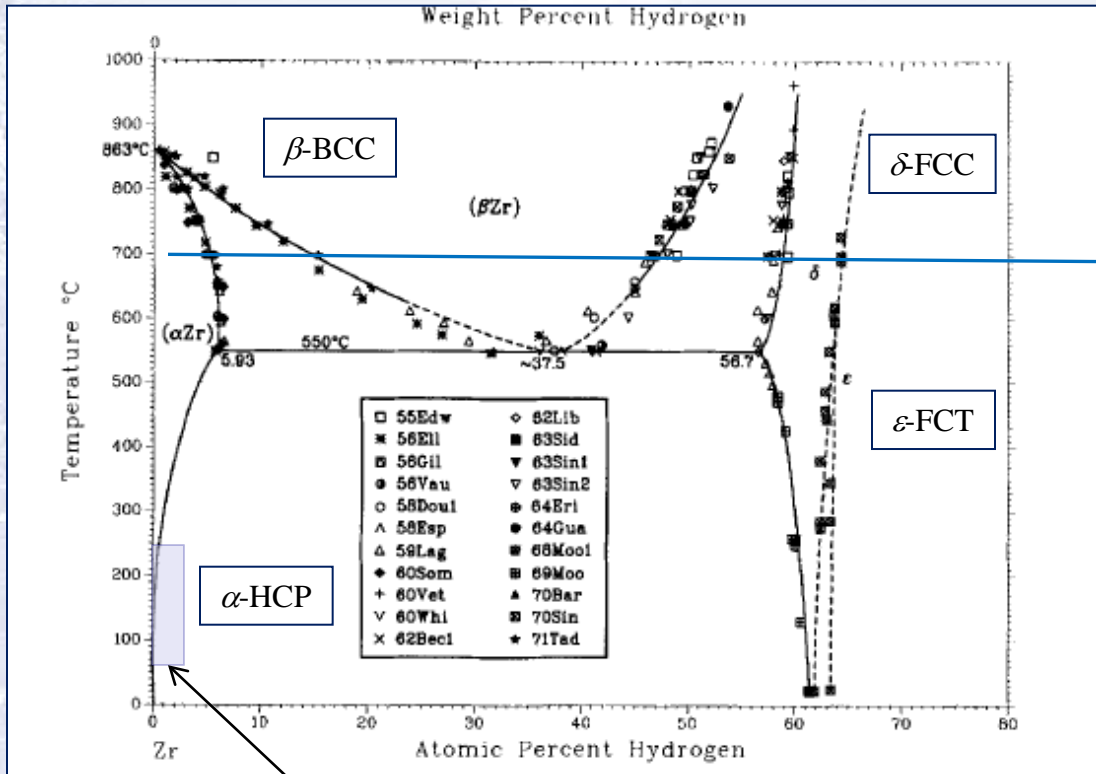
Partner	Year 1	Year 2	Year 3	Total
U. Illinois	\$538,764	\$543,416	\$557,768	\$1,639,945
U. Michigan	\$230,000	\$230,000	\$230,000	\$690,000
U. Florida	\$213,180	\$165,492	\$161,329	\$540,000
INL	\$150,000	\$150,000	\$150,000	\$450,000
ATI Wah Chang	\$60,000	\$60,000	\$60,000	\$180,000
U. Manchester (funded via the U.K. Research Council Energy Programme)				£984,270*
TOTALS	\$1,191,944	\$1,148,908	\$1,159,097	\$3,499,945*



Zr-Sn phase diagram



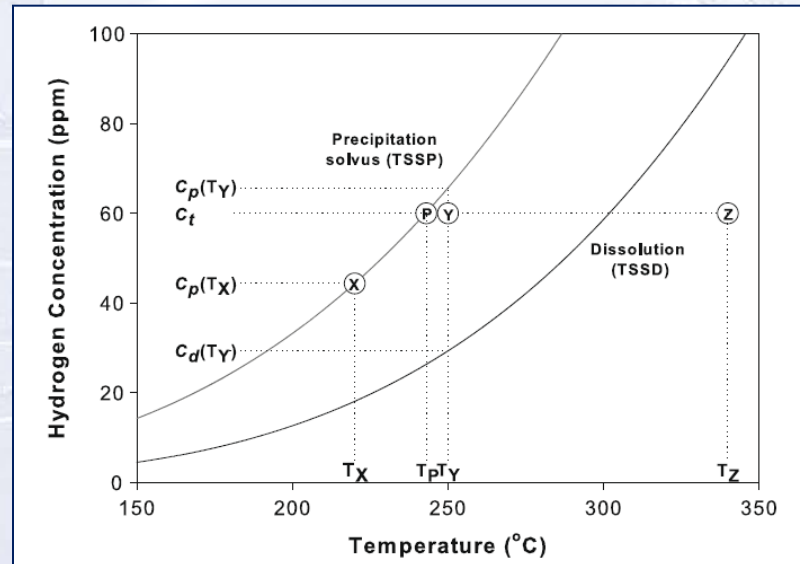
Zr-H and Zr-H-O Phase Diagrams



Relevant region for in-service and UFS



Zr-H phase behavior at low H concentration



G.A. McRae et al./Journal of Nuclear Materials 396 (2010) 130–143



Conclusions

- Multidisciplinary Approach to Accident Tolerant Fuel Cladding – materials, neutronics, irradiation performance, processing, corrosion, ...
- Response to large amounts of H generation due to high temperature oxidation of Zr-base cladding
- A major concern both Post-Three Mile Island and Post-Fukushima
- International Collaborations

